
Introduction of FIRE

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for the FIRE Study Team

Overview
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<http://fire.pppl.gov>

FIRE

Lighting the Way to Fusion



Fusion Science Objectives for a Major Next Step Burning Plasma Experiment

Explore and understand the strong non-linear coupling that is fundamental to fusion-dominated plasma behavior (self-organization)

- Energy and particle transport (extend confinement predictability)
 - Macroscopic stability (β -limit, wall stabilization, NTMs)
 - Wave-particle interactions (fast alpha particle driven effects)
 - Plasma boundary (density limit, power and particle flow)
- Test/Develop techniques to control and optimize fusion-dominated plasmas.
 - Sustain fusion-dominated plasmas - high-power-density exhaust of plasma particles and energy, alpha ash exhaust, study effects of profile evolution due to alpha heating on macro stability, transport barriers and energetic particle modes.
 - Explore and understand various advanced operating modes and configurations in fusion-dominated plasmas to provide generic knowledge for fusion and non-fusion plasma science, and to provide a foundation for attractive fusion applications.

Advanced Burning Plasma Exp't Requirements

Burning Plasma Physics

Q ≥ 5 , ~ 10 as target, ignition not precluded

$f_\alpha = P_\alpha/P_{\text{heat}}$ $\geq 50\%$, $\sim 66\%$ as target, up to 83% at $Q = 25$

TAE/EPM stable at nominal point, able to access unstable

Advanced Toroidal Physics

$f_{\text{bs}} = I_{\text{bs}}/I_p$ $\geq 50\%$ up to 75%

β_N ~ 2.5 , no wall ~ 3.6 , $n = 1$ wall stabilized

Quasi-stationary Burn Duration

Pressure profile evolution and burn control $> 10 \tau_E$

Alpha ash accumulation/pumping $> \text{several } \tau_{\text{He}}$

Plasma current profile evolution $1 \text{ to } 3 \tau_{\text{skin}}$

Divertor pumping and heat removal $\text{several } \tau_{\text{divertor}}$

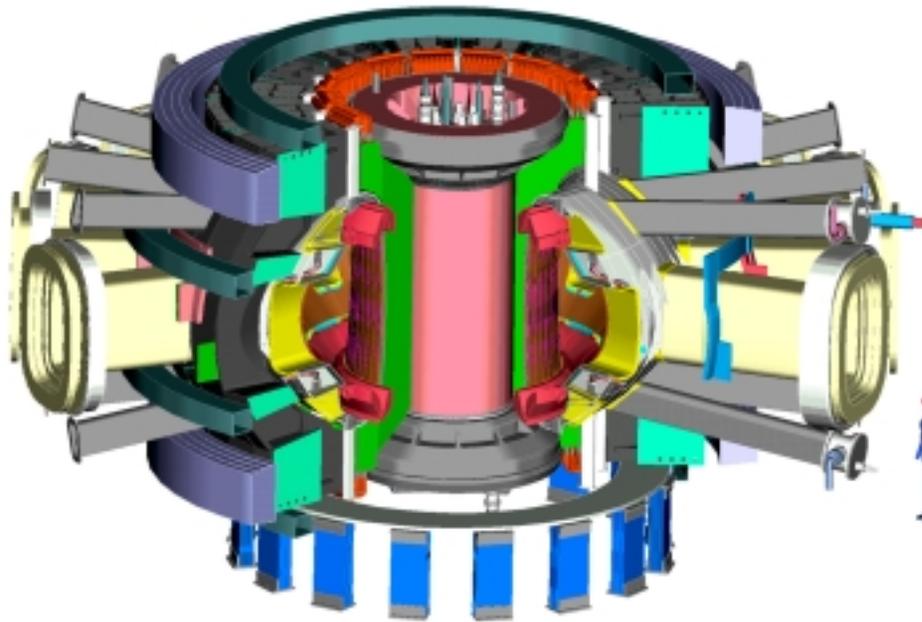
FIRE has Adopted the Advanced Tokamak Features Identified by ARIES Studies

- High toroidal field
- Double null
- Strong shaping
 - $\kappa = 2.0, \delta = 0.7$
- Internal vertical position control coils
- Cu wall stabilizers for vertical and kink instabilities
- Very low ripple (0.3%)
- ICRF/FW on-axis CD
- LH off-axis CD
- LHCD stabilization of NTMs
- Tungsten divertor targets
- Feedback coil stabilization for Resistive Wall Modes (RWM)
- Burn times exceeding current diffusion times
- Pumped divertor/pellet fueling/impurity control to optimize plasma edge

Fusion Ignition Research Experiment

(FIRE)

<http://fire.pppl.gov>



1,400 tonne

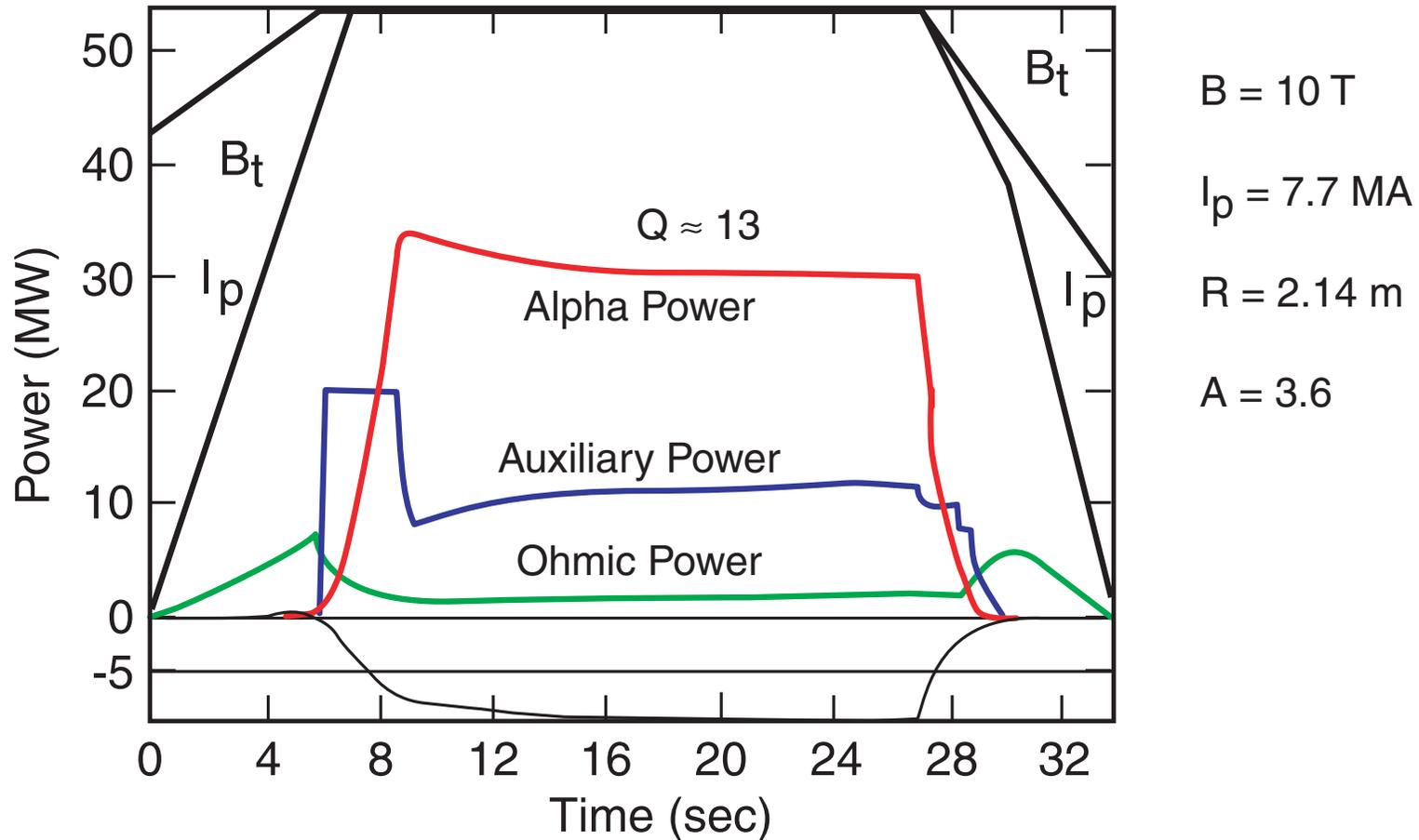
Design Features

- $R = 2.14 \text{ m}$, $a = 0.595 \text{ m}$
- $B = 10 \text{ T}$
- $W_{\text{mag}} = 5.2 \text{ GJ}$
- $I_p = 7.7 \text{ MA}$
- $P_{\text{aux}} \leq 20 \text{ MW}$
- $Q \approx 10$, $P_{\text{fusion}} \sim 150 \text{ MW}$
- Burn Time $\approx 20 \text{ s}$ ($2 \tau_{\text{cr}}$)
- Tokamak Cost $\approx \$351\text{M}$ (FY02)
- Total Project Cost $\approx \$1.2\text{B}$ (FY02)
at Green Field site.

Mission: Attain, explore, understand and optimize magnetically-confined fusion-dominated plasmas.

CIT + TPX = FIRE leading to ARIES

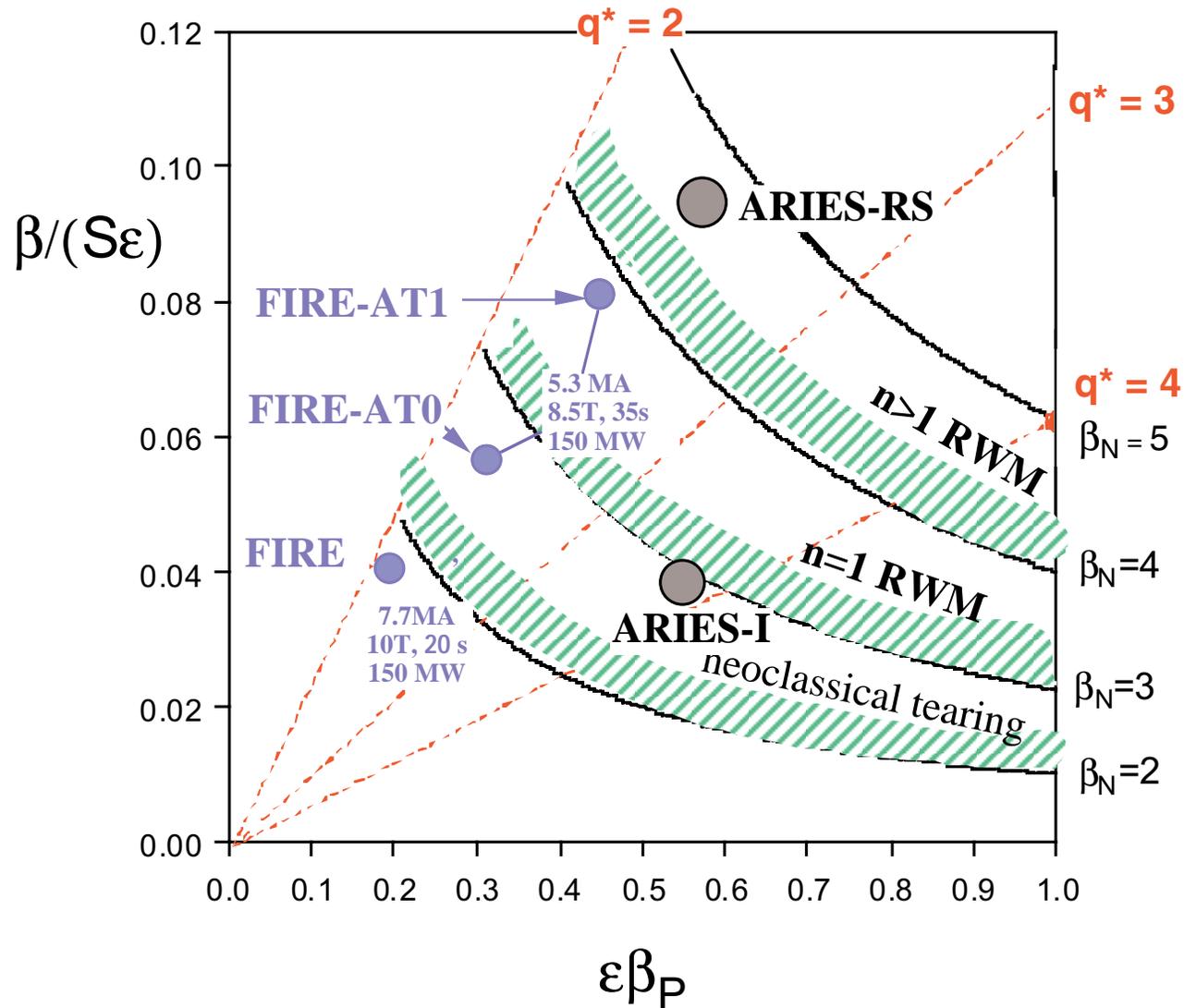
Simulation of Burning Plasma in FIRE



- ITER98(y, 2) with $H(y, 2) = 1.1$, $n(0)/\langle n \rangle = 1.2$, and $n/n_{GW} = 0.67$
- Burn Time $\approx 20 \text{ s} \approx 21\tau_E \approx 4\tau_{He} \approx 2\tau_{CR}$

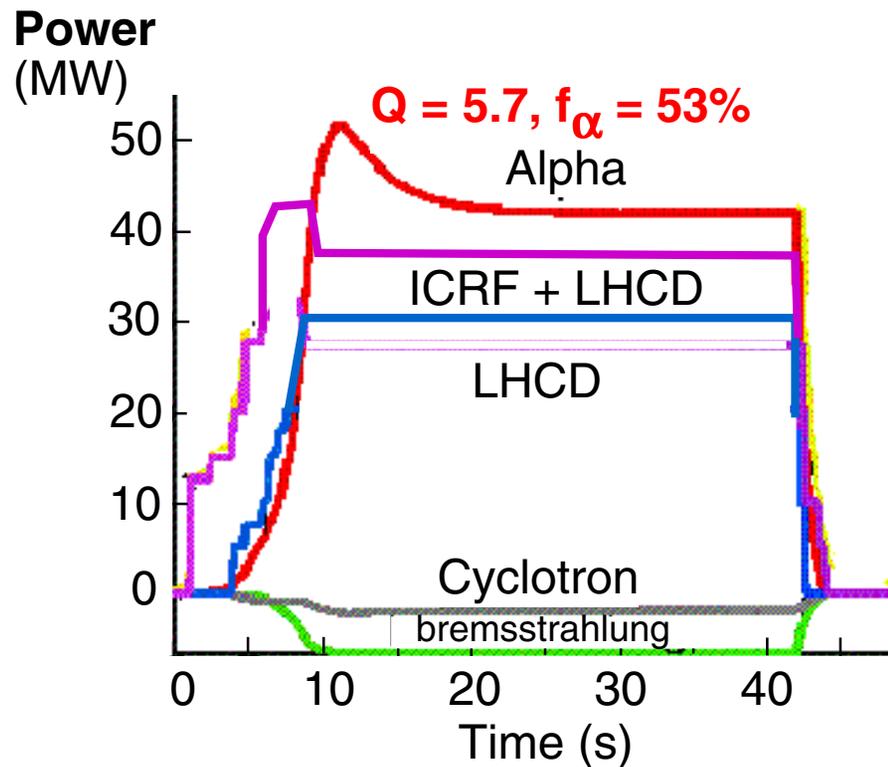
$$Q = P_{\text{fusion}} / (P_{\text{aux}} + P_{\text{oh}})$$

FIRE would Test a Sequence of AT Modes



Advanced Burning Plasma Physics could be Explored in FIRE

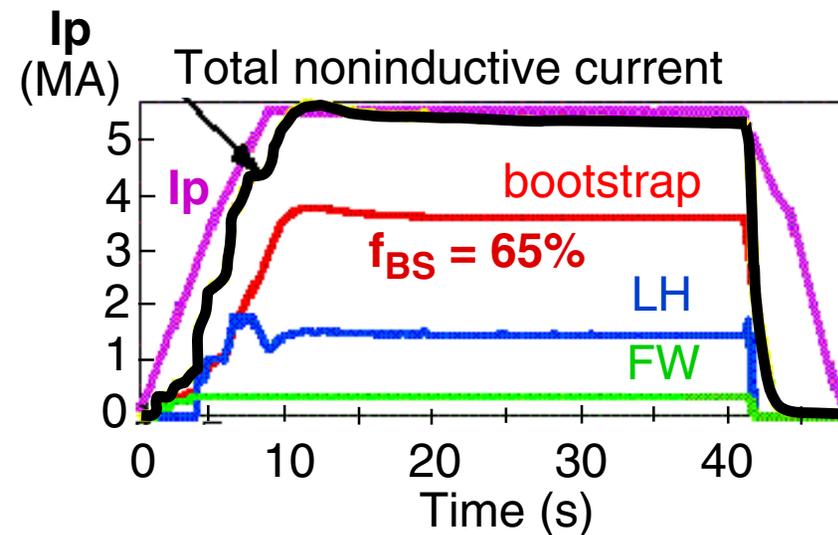
Self-Heating Dominant



Self-Current Drive Dominant

Fully Non-Inductive for $> 1 \tau_{CR}$

8.5 T, 5.4 MA, $t(\text{flattop}) = 32 \text{ s}$

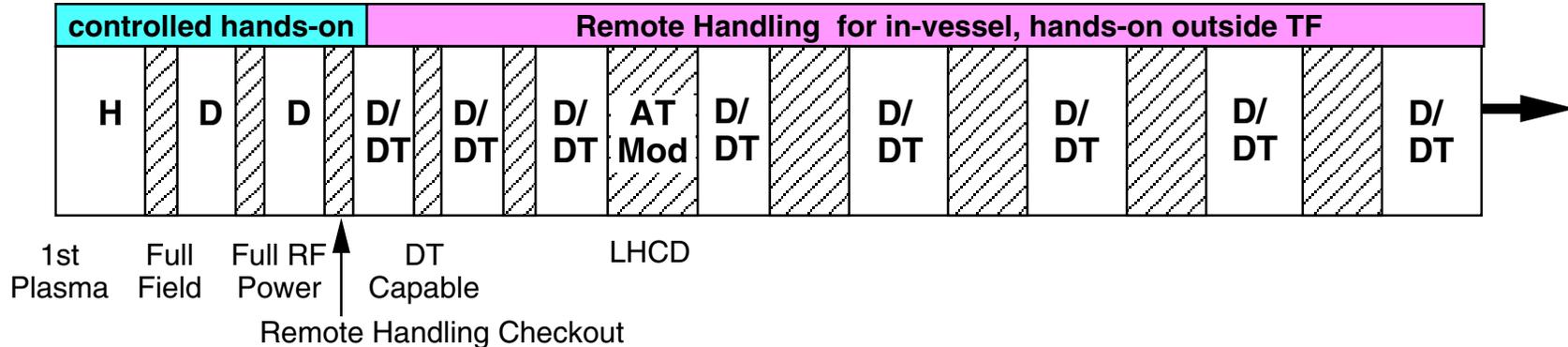


Tokamak simulation code results for $H(y, 2) = 1.4$, $\beta_N = 3.5$, would require RW mode stabilization. $q(0) = 2.9$, $q_{\min} = 2.2$ @ $r/a = 0.8$, 8.5 T, 5.5 MA

FIRE Experimental Plan

0	2	4	6	8	10	12	14	16	
Years from 1st plasma									
Shots/ 2yr	4000	4000	4000	4000	3500	3500	3500	3500	Original* Limits
Full B Shots/ 2yr	250	500	600	500	300	300	300	300	30,000
DT Energy(GJ)/ 2yr			1000	1000	1000	1000	1500	1000	3,000
Tritium Burnup(g)/2yr			2	2	2	2	3	2	6,500

Q~ 5 -10 (short pulse initially, extend to full power and pulse length)



- Control
- Cleanup
- Fueling
- Diagnostics
- Operations
- RF tests
- InitialRF Heating
- Plasma Power Handling
- Initial Physics studies
- Alpha heating
- Energy transport
- Fast particle
- Particle and ash removal
- Global Burn control
- Transient Profile control
- Transient Adv Tok
- Optimization of AT modes
- Non Inductive Profile control
- Improve Divertor and FW power handling
- Extend pulse length